

Design of Passive and Structural Conductors for Tokamaks Using Thin-Wall Eddy Current Modeling

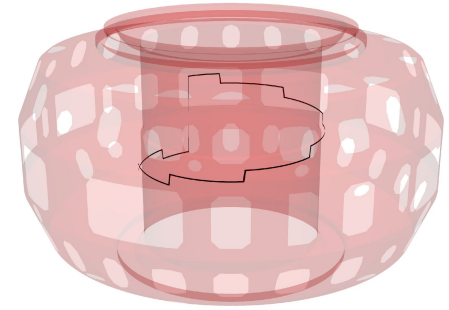
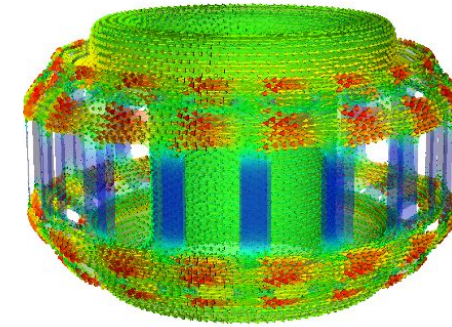
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D. Weisberg³, C. Paz-Soldan¹, R. Sweeney⁴,
R.A. Tinguely², A.J. Creely⁴

1- *Columbia University*

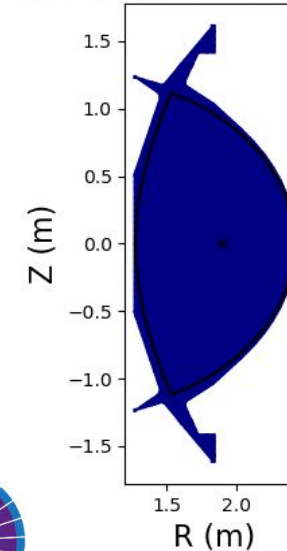
2- *MIT*

3- *General Atomics*

4- *Commonwealth Fusion Systems*



REMC $B_{r,n1}(G)$, $t=0.00ms$

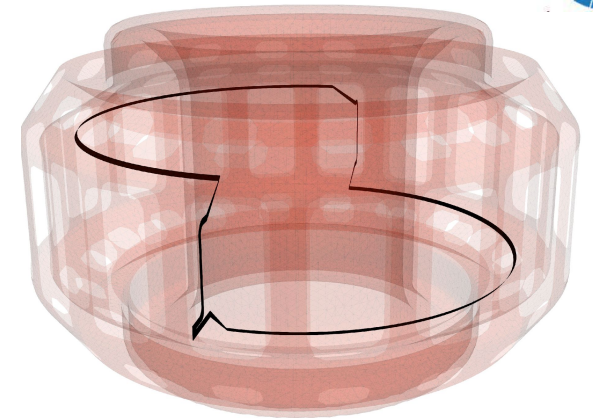
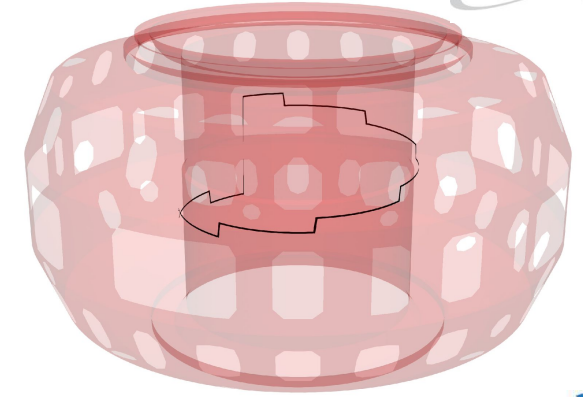


3D eddy currents can be an important consideration for tokamak design and engineering

- Large eddy currents are induced in tokamak conducting structures during both start-up and plasma disruptions
- These eddy currents can produce large non-axisymmetric fields which can:
 - Affect magnetic field null for start-up
 - Induce large forces on conducting components
 - Drive dangerous MHD instabilities
- Conducting paths can also be explicitly designed to generate desirable 3D fields, such as the proposed Runaway Electron Mitigation Coils (REMCs)
- The ability to model these conductors and their interactions with high fidelity early in the design process is important capability for tokamak engineering

Runaway Electron Mitigation Coils (REMC) couple to loop voltage produced by disruption current quench

- **Runaway electrons (RE) are generated during a disruption current quench (CQ)**
 - RE avalanche drives exponential growth ($\sim I_p$)
- **Passive non-axisymmetric coil designed to inductively couple to changing plasma current**
 - Current driven by large toroidal electric field created during CQ
- **If properly designed the coil will produce large 3D field during CQ**
 - Will destroy confining flux surfaces and drive large deconfining MHD
 - Both DIII-D and SPARC REMC are designed to create n=1 radial field
 - Modeling and experiment have demonstrated the efficacy of 3D field in deconfining REs



*** Talks by Izzo and Sweeney**

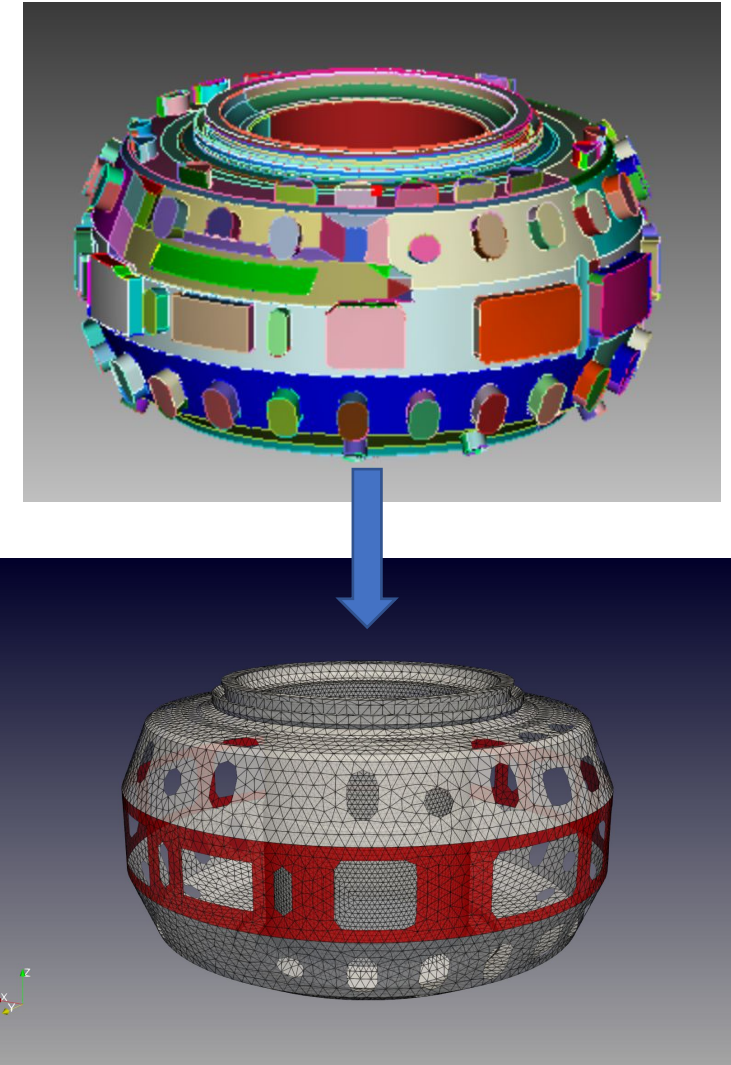
Boozer, *PPCF* **53** 084002 (2011)
Mlynar, *PPCF* **61** 014010 (2019)
Paz-Soldan, *PPCF* **61** 054001 (2019)
Tinguely, *Nucl. Fusion* **61** 124003 (2021)
Weisberg, *Nucl. Fusion* **61** 106033 (2021)
Izzo, *Nucl. Fusion* **62** 096029 (2022)

Outline

- Motivation
- The ThinCurr 3D thin-wall E-M code
- Runaway Electron Mitigation Coil modeling for DIII-D and SPARC
 - Coil to vessel separation (DIII-D only)
 - Coil resistance
 - Current quench duration
 - Vertical displacement
- Design of REMC-like coil for validation on HBT-EP
- Conclusions and future work

ThinCurr is a new 3D thin-wall eddy current modeling tool based on the PSI-Tet 3D MHD code

- Workflow begins with CAD model of tokamak conducting structures (VV, etc.)
 - Geometry is reduced to sheet representation (thin-wall limit)
- These models are then defeatured to optimize computation
 - Important details such as port holes, stability plates, etc. are retained
- The surfaces are then meshed to the desired resolution
 - Different components can have higher/lower resolution
- Different materials and thickness can all be captured in a single model
 - DIII-D model has 4 resistivities
 - SPARC model has 6+ resistivities

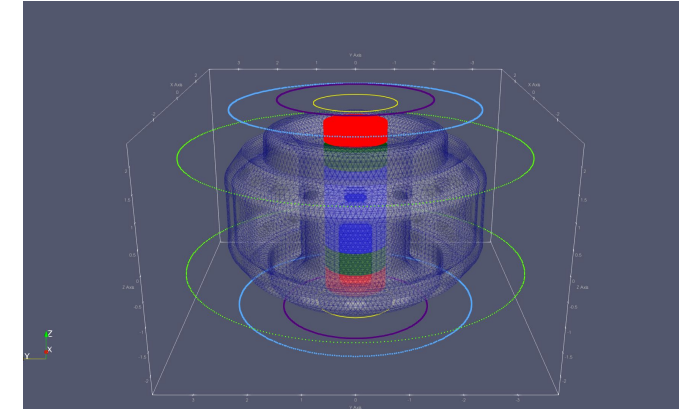


Minicurl models 3D conducting structures as a series of coupled circuit equations discretized with a finite elements

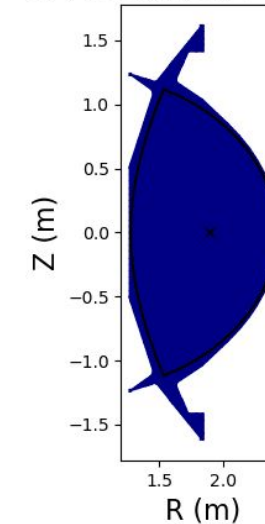
- **Physics governed by inductance/resistance**

$$L_{i,j} \frac{\partial I_i}{\partial t} + R_{i,j} I_i = V_j(t)$$

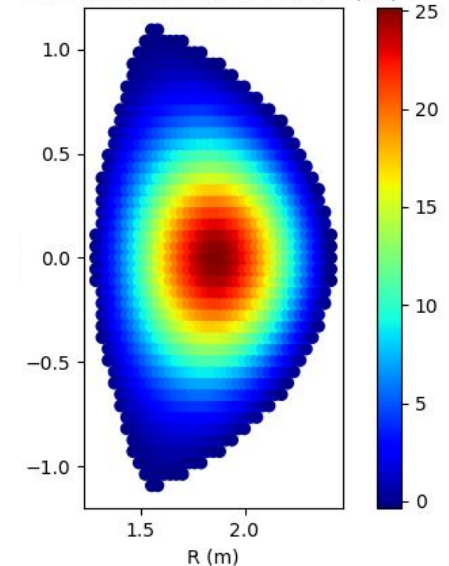
- **Additional currents/voltages can be included**
 - Filament coils ($I(t)$ or $V(t)$)
 - Plasma “modes” (eg. DCON)
- **Magnetic fields available anywhere in space**
 - Sensor signals with eddy currents
 - Lorentz forces on structures
 - Discontinuities in $\mathbf{B} \times \hat{\mathbf{n}}$ at surfaces



REMC $B_{r,n1}(G)$, $t=0.00ms$



Current in Plasma Filaments (kA)

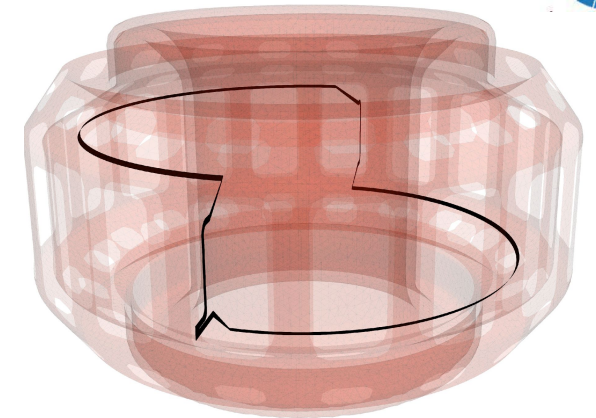
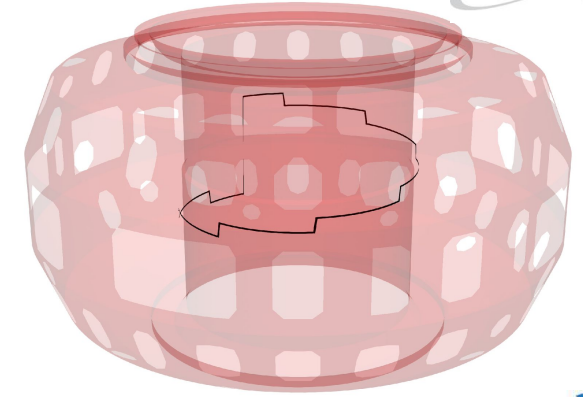


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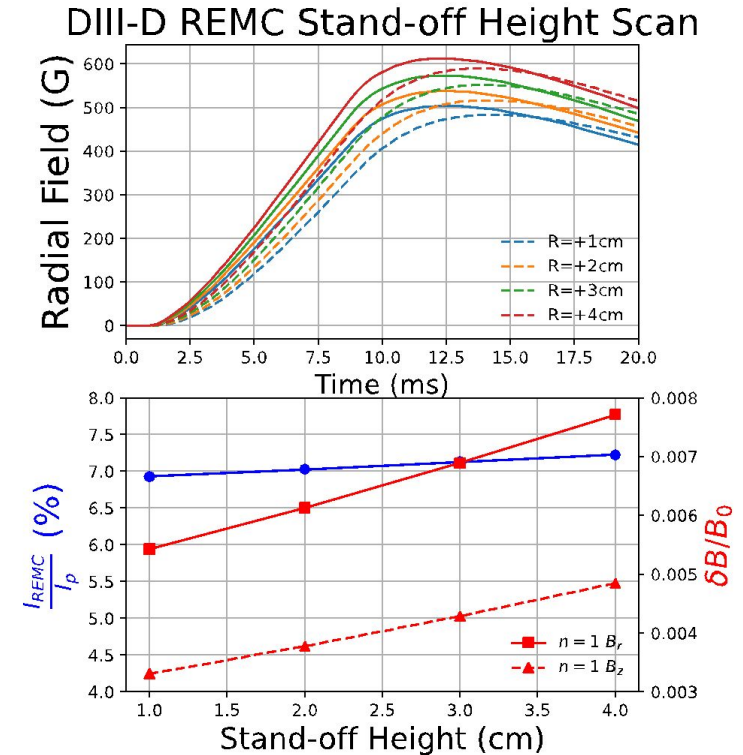
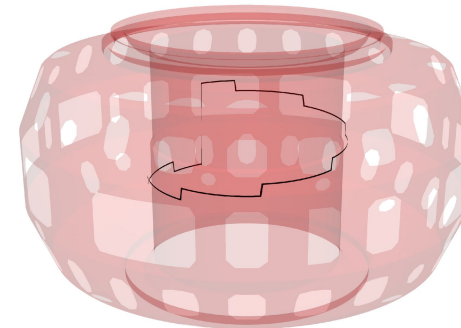
Multiple studies were completed to assess effectiveness and inform the design of DIII-D and SPARC REMCs

- REMC designs modeled using the ThinCurr code
- Simulated with fully 3D REMC models
 - Coil modeled as thin sheet with variable resistor
 - Toroidally flowing current measured using “jumper”
- Effect of standoff, variable resistor value, CQ time, and vertical plasma position
- Publication on results: A. Battey, et al. *under review*



DIII-D's coil location makes it sensitive to position relative to the center stack structure

- Eddy currents induced in the center stack significantly slow the time-response
 - Primarily due to image currents that cancel the coil field
- Maximum change in induced current: 4.1%
- Maximum change in applied radial field: 29.6%
- Maximum change in applied vertical field: 31.7%

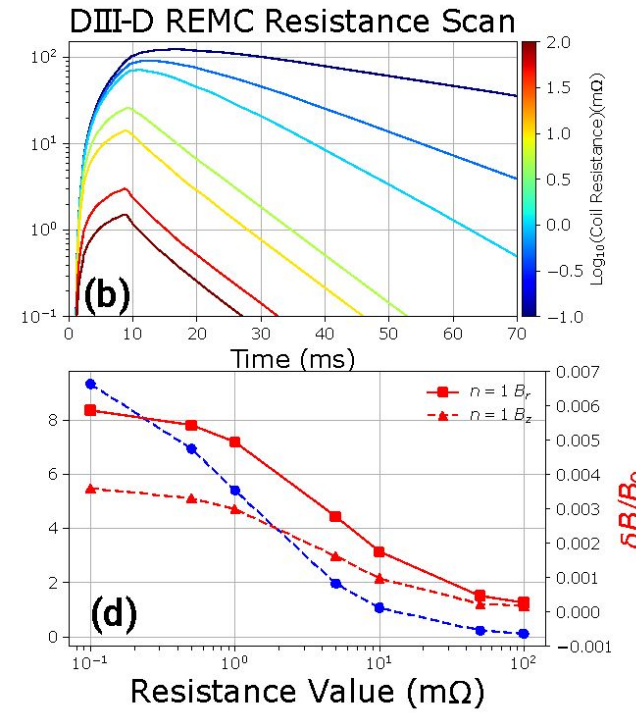
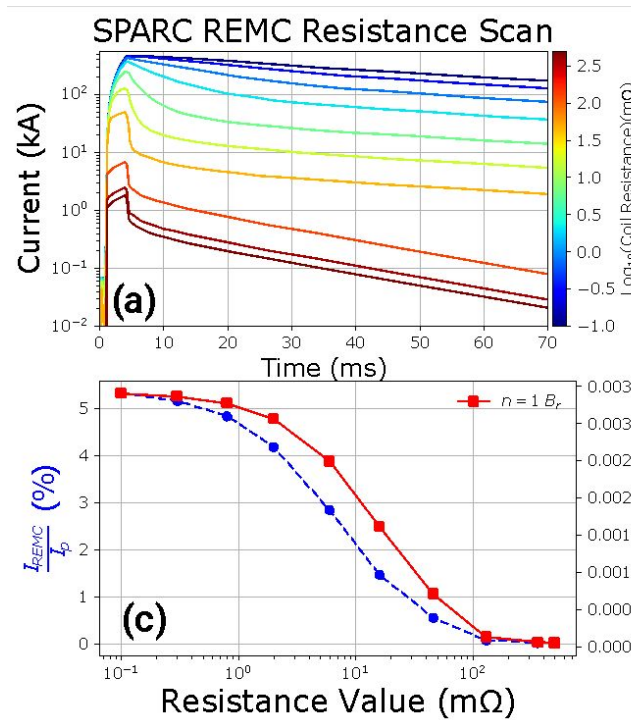


$$\tau_{CQ} \approx 9 \text{ ms}$$

External resistors can be added to limit total impulse, but can also reduce REMC-produced field during CQ



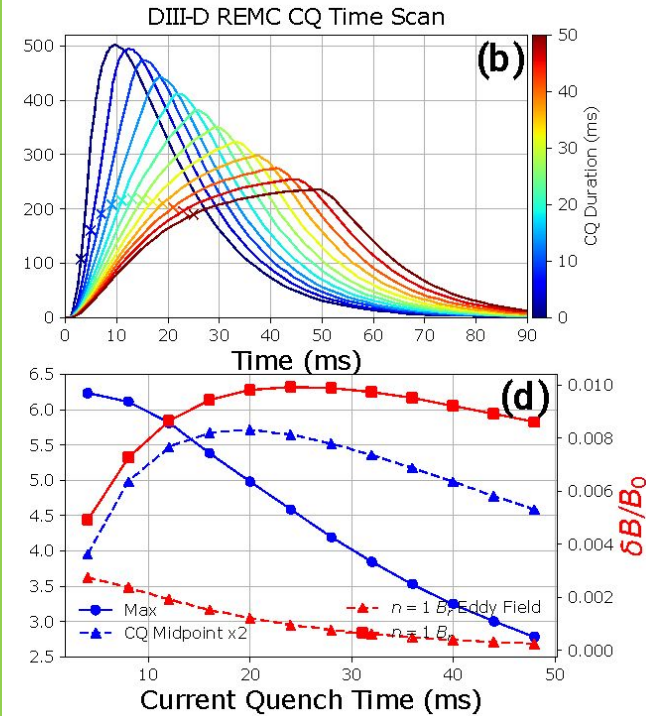
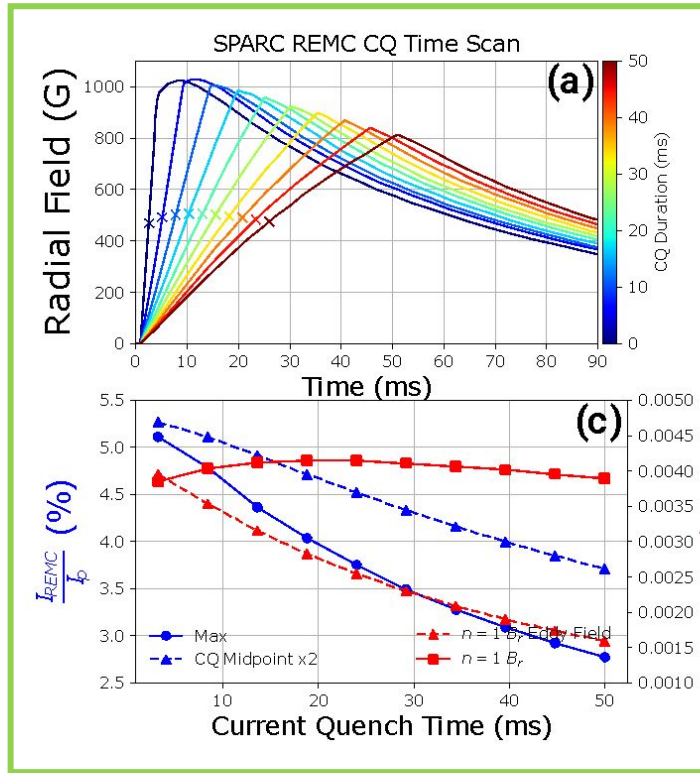
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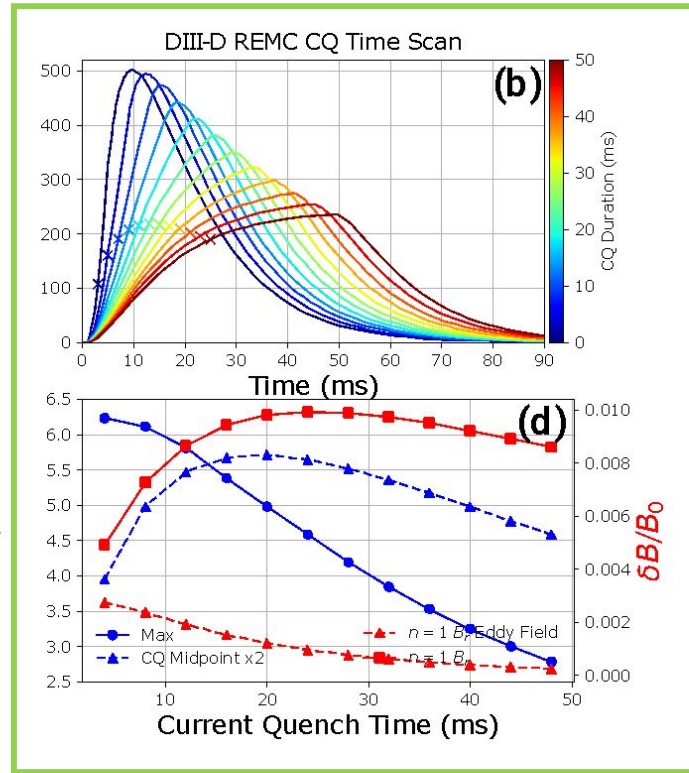
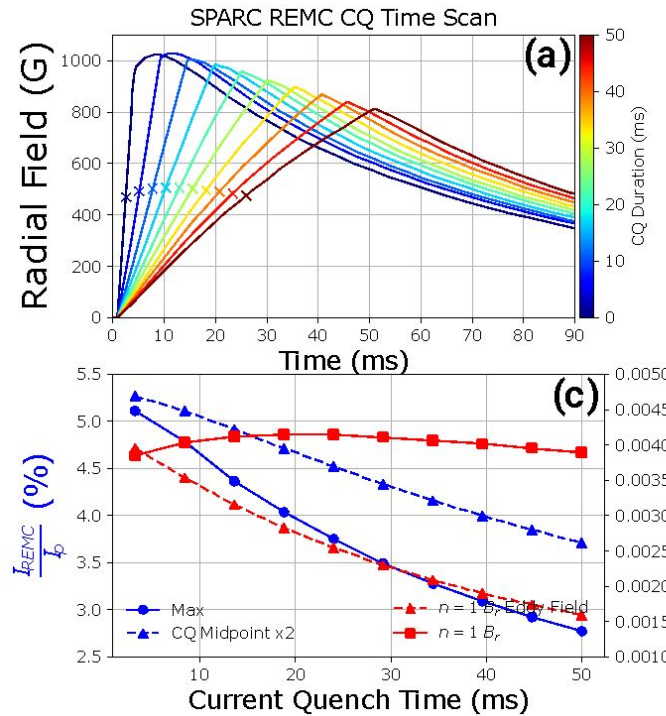
- Primarily impacts current mid to late in CQ (inductively limited early in time)
- On SPARC longer characteristic vessel times maintain currents for longer

REMC-produced field stays fairly constant across expected current quench durations in SPARC and DIII-D



- For SPARC, the applied field depends only weakly on CQ length
 - Current induced in the coil varies by 43%
 - Eddy currents limit the applied field more for fast quenches, leading to a nearly constant value (5%)

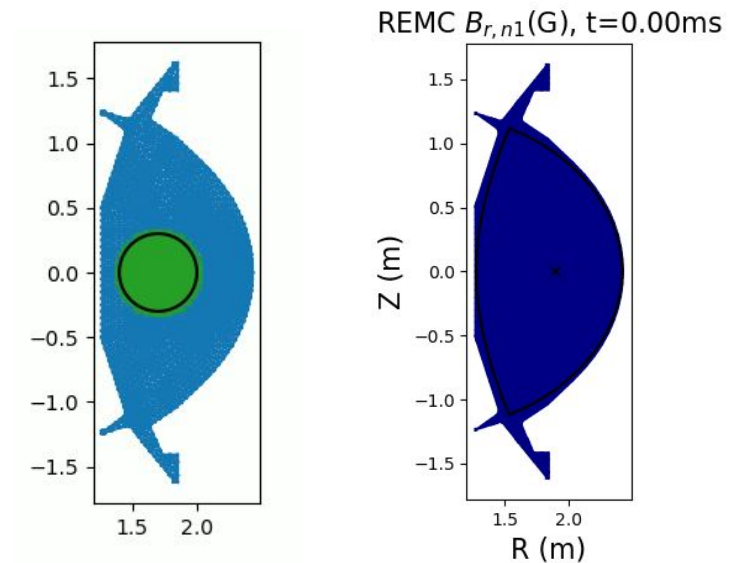
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- **DIII-D REMC field is lower for the fastest CQ times, before plateauing**
 - Peak in applied field occurs at intermediate (larger than common experimental) CQ times
 - Caused by strong reduction in effective plasma-REMC coupling on fast time scales (due to VV)

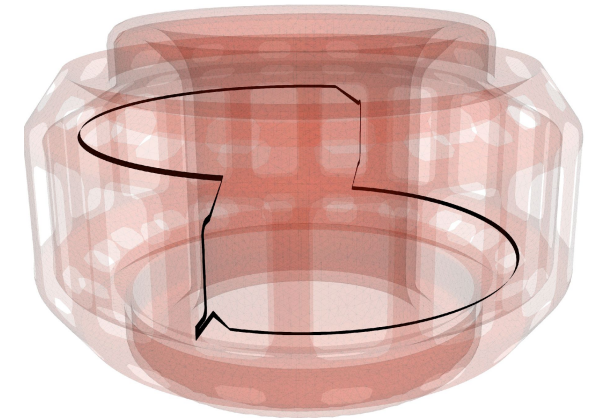
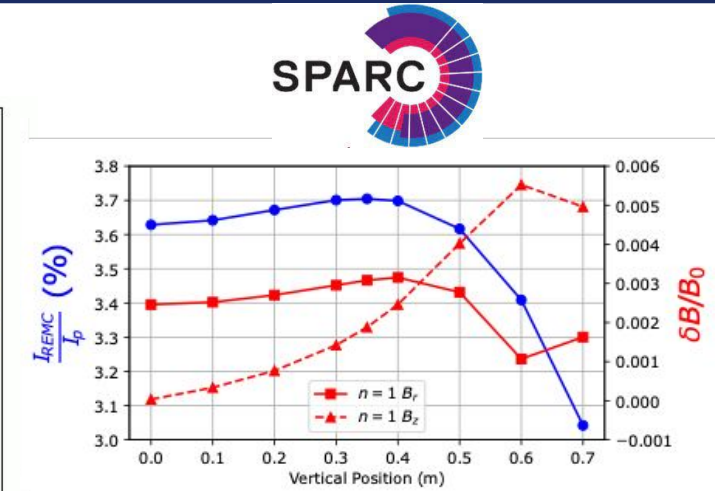
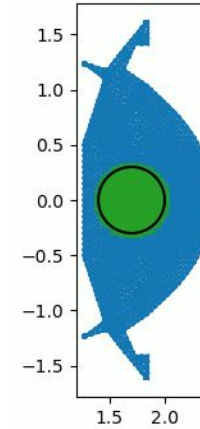
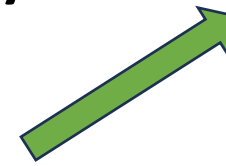
For VDEs the coupling to the REMC and the magnitude of the REMC-produced field on the plasma changes in time

- Important for REMCs to be robust to vertically unstable plasmas
 - Performance evaluated for vertically shifted plasma
- **Circular cross section plasma shifted vertically**
 - CQ always modeled with a stationary position



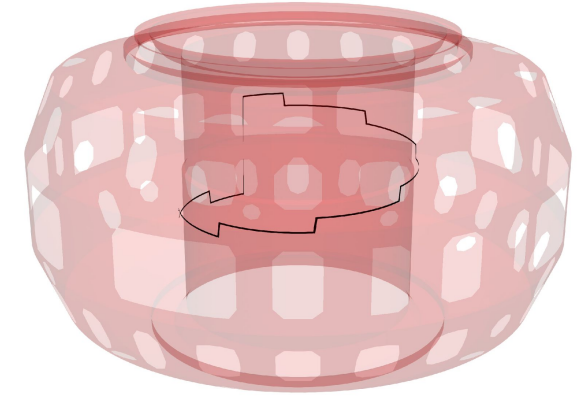
The SPARC and DIII-D designs exhibit different changes in REMC effectiveness during VDEs

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- While SPARC REMC applies a slightly weaker radial field its vertical field quickly grows
 - Due to position of coil on upper low-field side
 - More than compensates for decreasing radial field

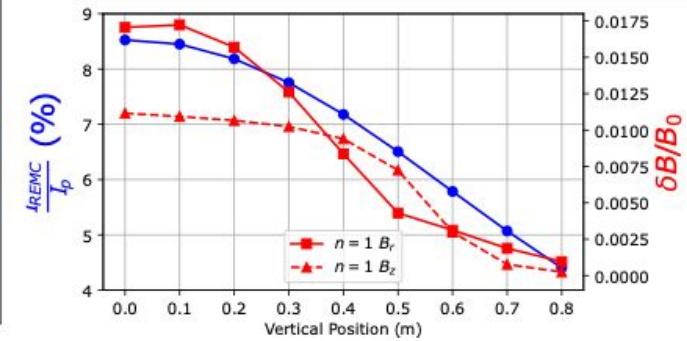
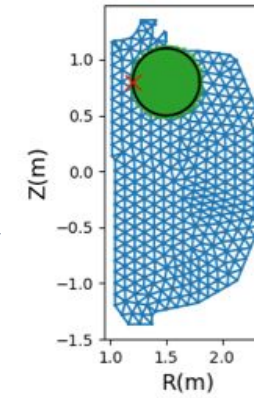


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- By contrast the performance of the DIII-D REMC coil quickly declines with ΔZ
 - Plasma moves uniformly away from coil

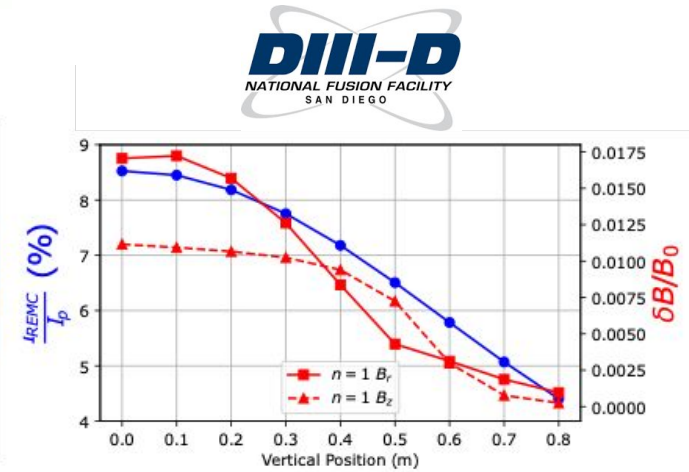
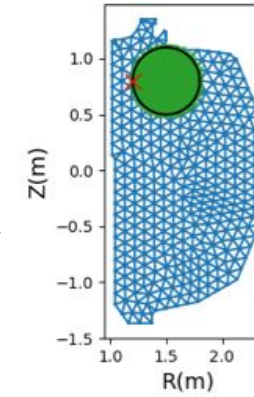
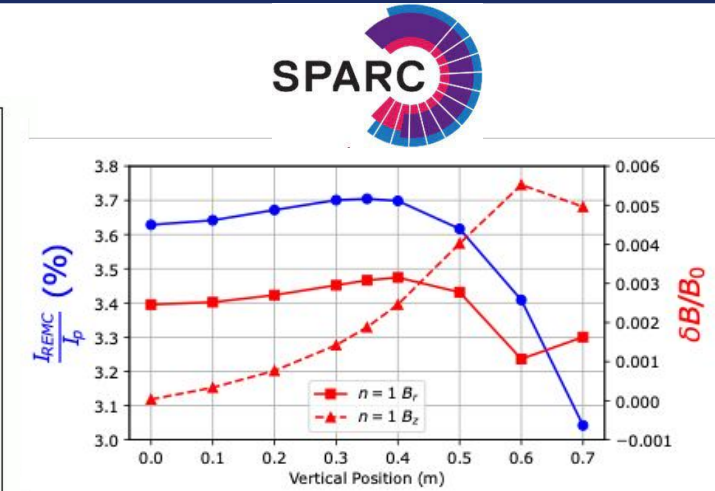
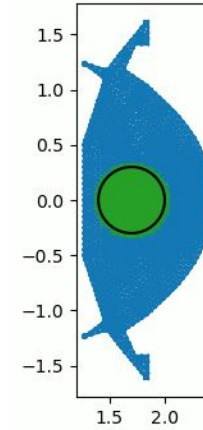


DIII-D
NATIONAL FUSION FACILITY
SAN DIEGO



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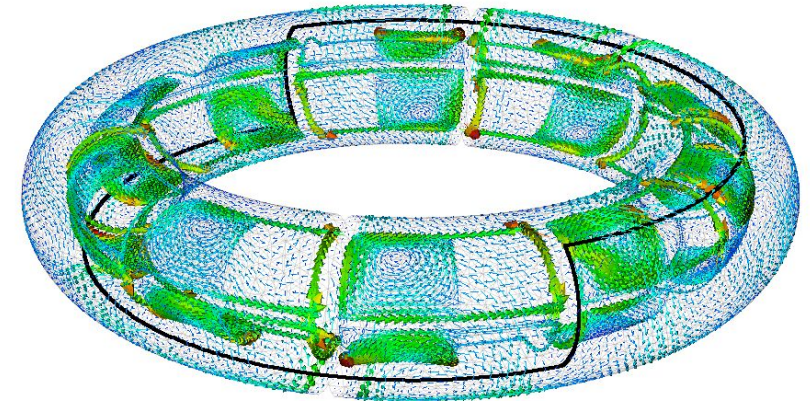
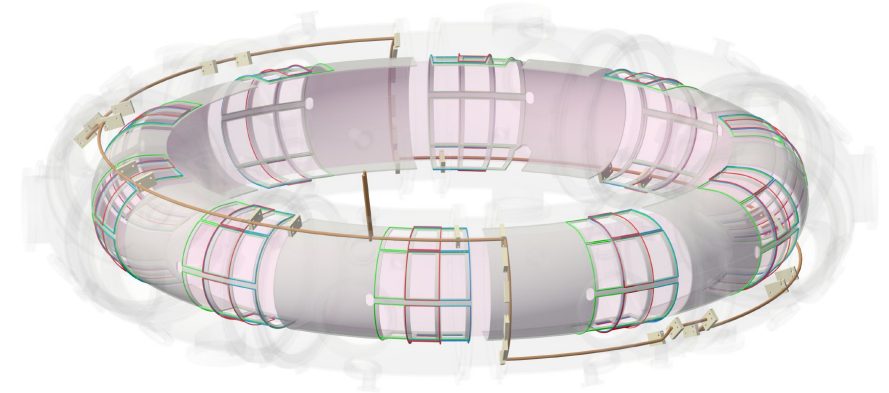
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An REMC-like coil has been proposed for HBT-EP to validate E-M and plasma response models



- **An REMC-like coil was designed for HBT-EP to validate ThinCurr and related models**
 - Focused on validating plasma-coil coupling
 - Install and test within 2 years
- **Coil can be passively driven (disruptions) or actively driven by external supplies**
 - Study plasma response
 - Interact with startup-generated runaways
- **Leverages unique features of HBT-EP**
 - Control coil array for baseline comparison

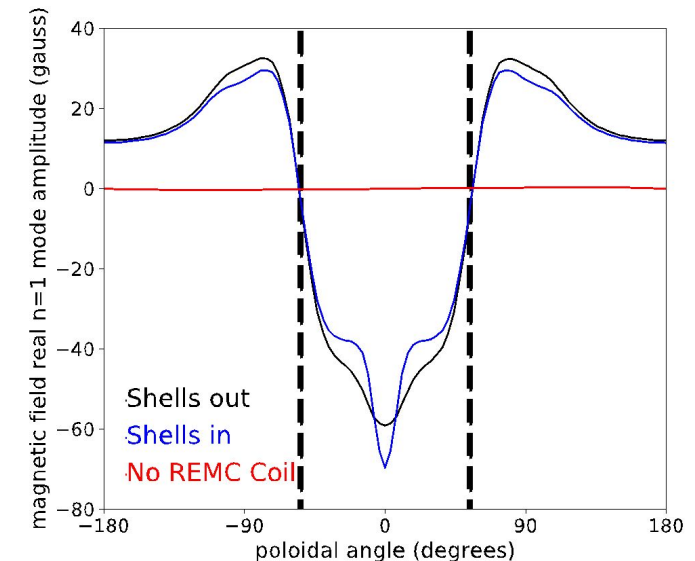
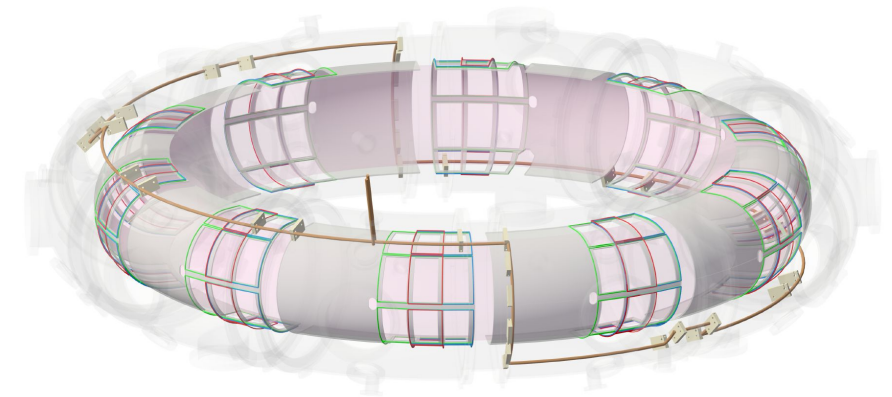


* Poster by Braun

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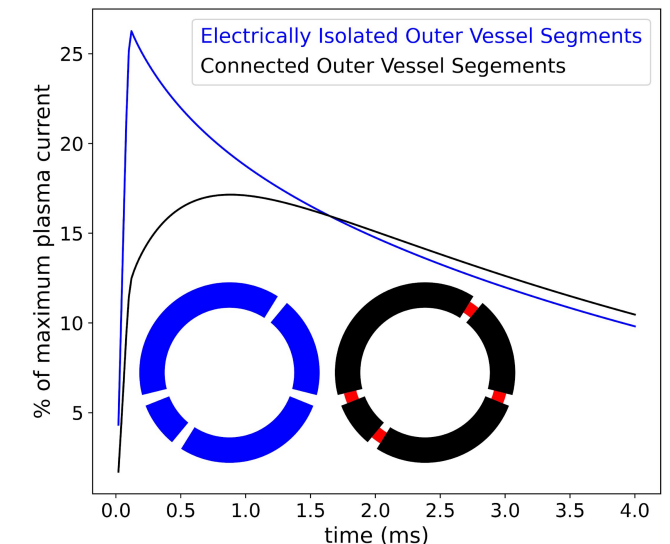
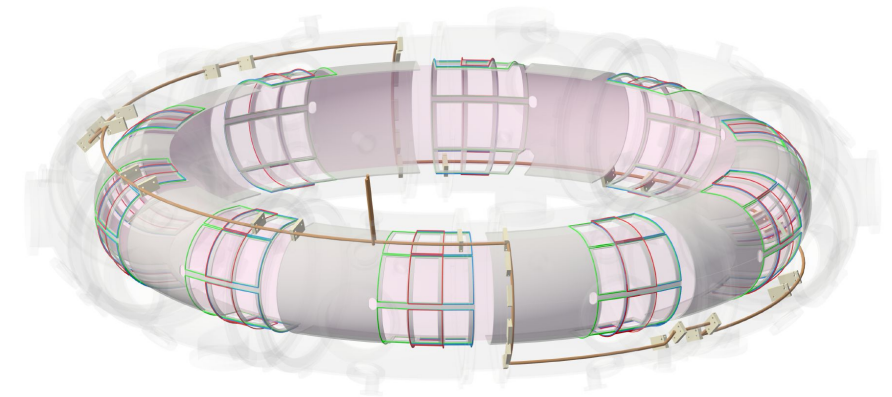
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 - High-resolution magnetics (fields and plasma response)
 - Moveable walls (field at plasma surface)



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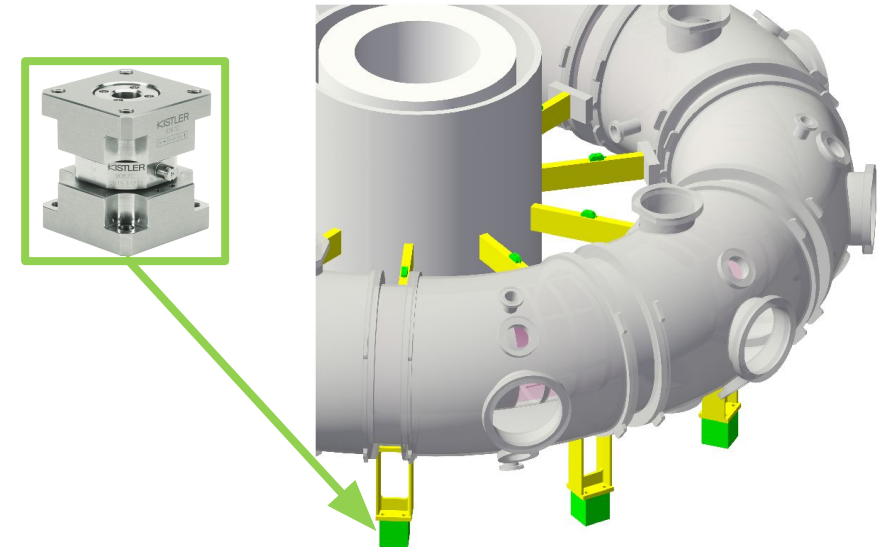
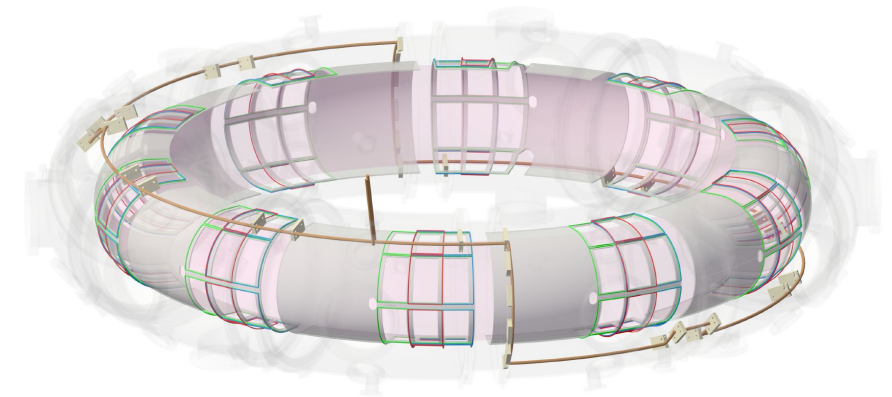
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 - Proposed structural force sensors



Conclusions

- The newly developed ThinCurr 3D E-M modeling code has been used to predict the behavior of REMC coils proposed for DIII-D, SPARC, and HBT-EP
 - Code is open-source and available for other applications as well (reach out to me)
- Standoff height was found to be particularly important for DIII-D REMC design
- Both DIII-D and SPARC coils exhibit only modest performance dependence on current quench duration
- For VDEs, the strength of the applied field appears more consistent for the SPARC design than the DIII-D design
- A coil has been designed and proposed for HBT-EP to enable the validation of E-M and some plasma response models for REMCs